

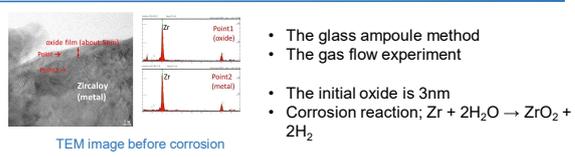
## SUMMARY

The long-term aqueous corrosion of irradiated and non-irradiated zirconium alloys was studied. A sensitive hydrogen measurement was selected for the non-irradiated alloy based on the reaction of  $Zr + 2H_2O \rightarrow ZrO_2 + 2H_2$ . The corrosion rates decreased with time and increased at a higher temperature, but the influences of pH and other chemical properties of the solution on the rate were not significant. The equivalent corrosion rates obtained from the leached <sup>14</sup>C were less than that of non-irradiated Zr alloys. The fraction of leached <sup>14</sup>C from irradiated Zircaloy-2 (BWR) as gas, dissolved organics and dissolved inorganics was obtained. The fraction in the liquid phase increased with time and reached over 90% after 2 years. The inorganic/organic ratios of the liquid were around 1/3 and seemed to be not depending on time. Instant release fraction (IRF) for spent claddings was also discussed. The inventory measurement shows that the abundance of <sup>14</sup>C in the oxide was only 7.5%, and the leached <sup>14</sup>C from the irradiated cladding with oxide was found to be 0.0038%. These understandings should be reflected in the future safety assessment so that a lower, or potentially negligible, IRF can be supported.

## OBJECTIVES

- Long-term corrosion rate of Zr alloys obtained by hydrogen (gas + absorbed), and affecting factors as temperature, pH, and Na/Ca, and congruence of <sup>14</sup>C release.
- Released <sup>14</sup>C fraction from irradiated cladding as gas, dissolved organics, and dissolved inorganics.
- Discussion on the instant release fraction (IRF) for irradiated cladding with regards to oxide inventory and <sup>14</sup>C release from oxide.

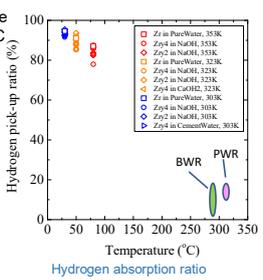
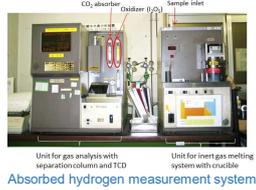
## Corrosion Test for Non-irradiated Zr alloys



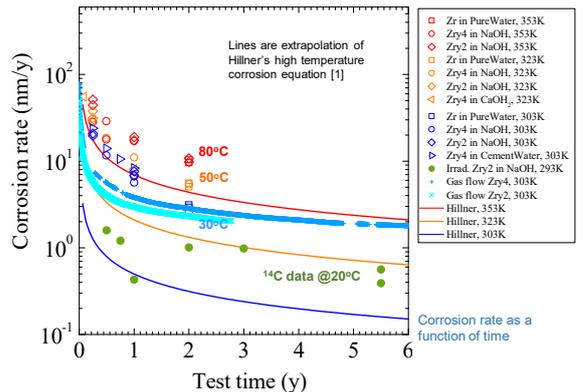
- The glass ampoule method
- The gas flow experiment
- The initial oxide is 3nm
- Corrosion reaction;  $Zr + 2H_2O \rightarrow ZrO_2 + 2H_2$

## Hydrogen Absorption

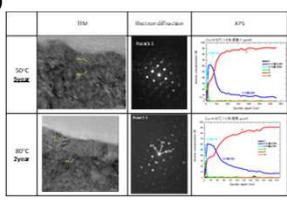
- Absorbed hydrogen was measured by the inert gas melting system coupled with GC (Japanese Industrial Standard Z 2614)
- Initial hydrogen content is 10 ppm



## Long-term Corrosion Rate



- Corrosion rate decreases with time (close to the Parabolic law [2])
- The data higher than Hillner's equation
- <sup>14</sup>C data lower than hydrogen data
- Oxide is well crystallized and the tetragonal structure can be confirmed



## CONCLUSIONS

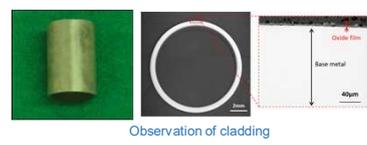
- Long-term corrosion rate decreases with time and is around 4 nm/y at 30°C (after 2 years).
- Leached <sup>14</sup>C from irradiated cladding is mainly dissolved organics.
- Instant release fraction (IRF) is suggested negligible or at least less than 20% assumed in the safe case.

## ACKNOWLEDGMENT:

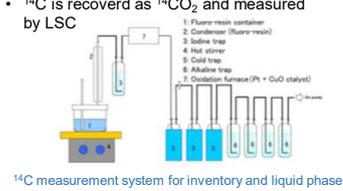
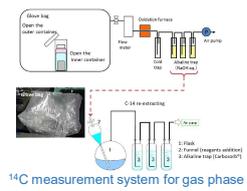
This research is a part of the "Research and development of processing and disposal technique for TRU waste" program funded by Agency for Natural Resources and Energy, Ministry of Economy, Trade and Industry of Japan.



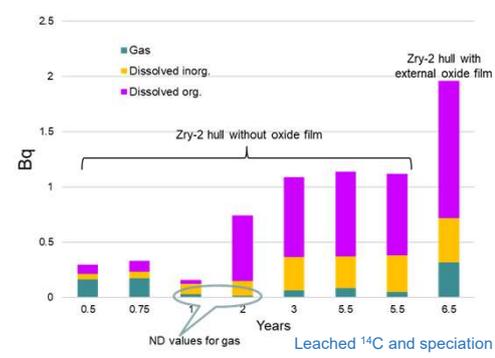
## Corrosion Test for Irradiated Zircaloy-2



- BWR Zircaloy-2 (Burnup; 41.6 and 39.7 GWd/t)
- Cladding with and without oxide film for immersion test
- Container of glass inner vial and gas tight outer SS vessel
- NaOH solution at pH 12.5
- Room temperature
- <sup>14</sup>C is recovered as <sup>14</sup>CO<sub>2</sub> and measured by LSC



## Leached <sup>14</sup>C

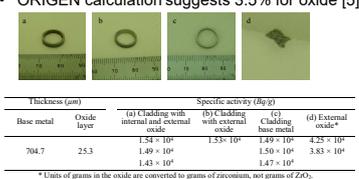


- The total leached <sup>14</sup>C is small up to 1- 2 Bq.
- Released <sup>14</sup>C as gas is a significant fraction (53-55%) during the first year. After 1 year, the gas fraction decreases.
- In the dissolved fraction, the organics are high proportion to the inorganics.
- For sample with oxide, the total leached <sup>14</sup>C is relatively higher than without oxide.
- The overall trend is similar between with and without oxide.
- Except data up to 1 year, the gas is a minor fraction.
- The dissolved organic is major around 60-70%.

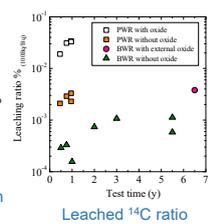
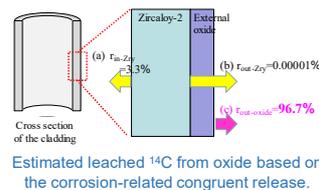
## Instant Release Fraction (IRF)

- Preliminary safety case in Japan [3]
- Hull oxide is regarded as a source of IRF and the oxide inventory is assumed to be 20% for hull waste

- <sup>14</sup>C specific activity for oxide is larger than metallic part
- But <sup>14</sup>C abundance for oxide is 7.5%
- ORIGEN calculation suggests 3.5% for oxide [5]



- Leaching experiment using cladding with oxide [4]
- Oxide should be a main leaching source
- However, leaching ratio (total leached Bq / inventory) is only 0.0038% after 6.5 years



## REFERENCES

- HILLNER, E. 1977. Corrosion of zirconium-base alloys—An overview, *Zirconium in the Nuclear Industry, 3rd Int. Symp., ASTM STP 633*, 211-235.
- SAKURAGI, T. et al. 2013. Long-term corrosion of Zircaloy-4 and Zircaloy-2 by continuous hydrogen measurement under repository condition, *Mater. Res. Soc. Symp. Proc.* 1518, 173-178.
- FEPC and JAEA 2007, Second Progress Report on Research and Development for TRU Waste Disposal in Japan, JAEA AND FEPC REPORT TRU-2.
- SAKURAGI, T. et al. 2016. Carbon 14 distribution in irradiated BWR fuel cladding and released carbon 14 after aqueous immersion of 6.5 years, *Procedia Chemistry* 21, 341-348.
- SAKURAGI, T. et al. 2013. Estimation of Carbon 14 Inventory in Hull and End-Piece Wastes from Japanese Commercial Reprocessing Operation, *Proceedings of the ICEM2013*. Brussels, Belgium.